

Assessment and interpretation of internal doses : uncertainty and variability

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ICRP C2

This presentation has neither been approved nor endorsed by the Main Commission of ICRP

Internal exposures are managed by the use of the committed effective dose

$$e(\tau) = \sum_T w_T \left[\frac{h_T^M(\tau) + h_T^F(\tau)}{2} \right]$$

Calculating committed effective dose after internal contamination is a complex procedure

Intake



**Deposition
in tissues**

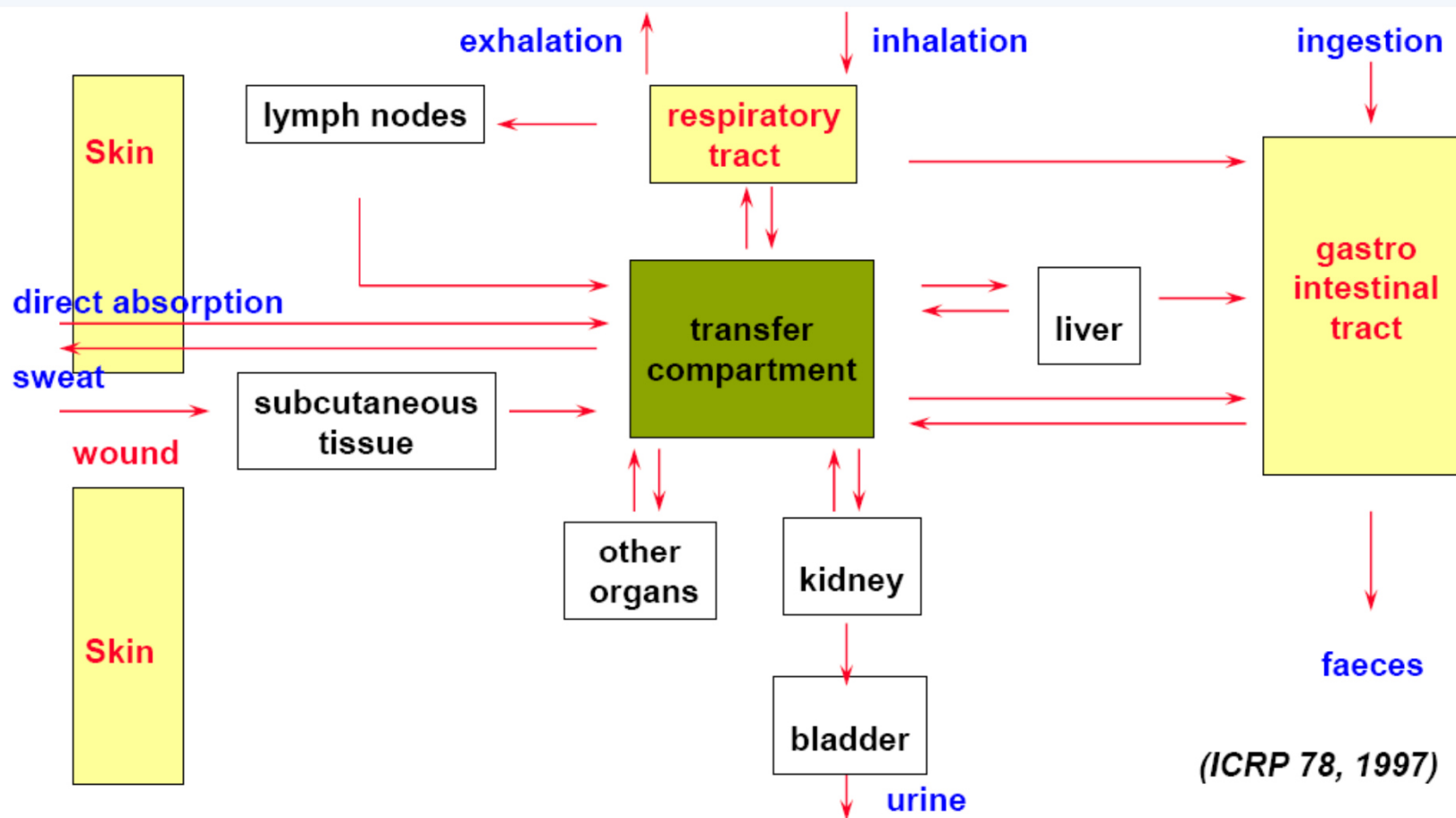
Biokinetic models

Intake



Deposition
in tissues

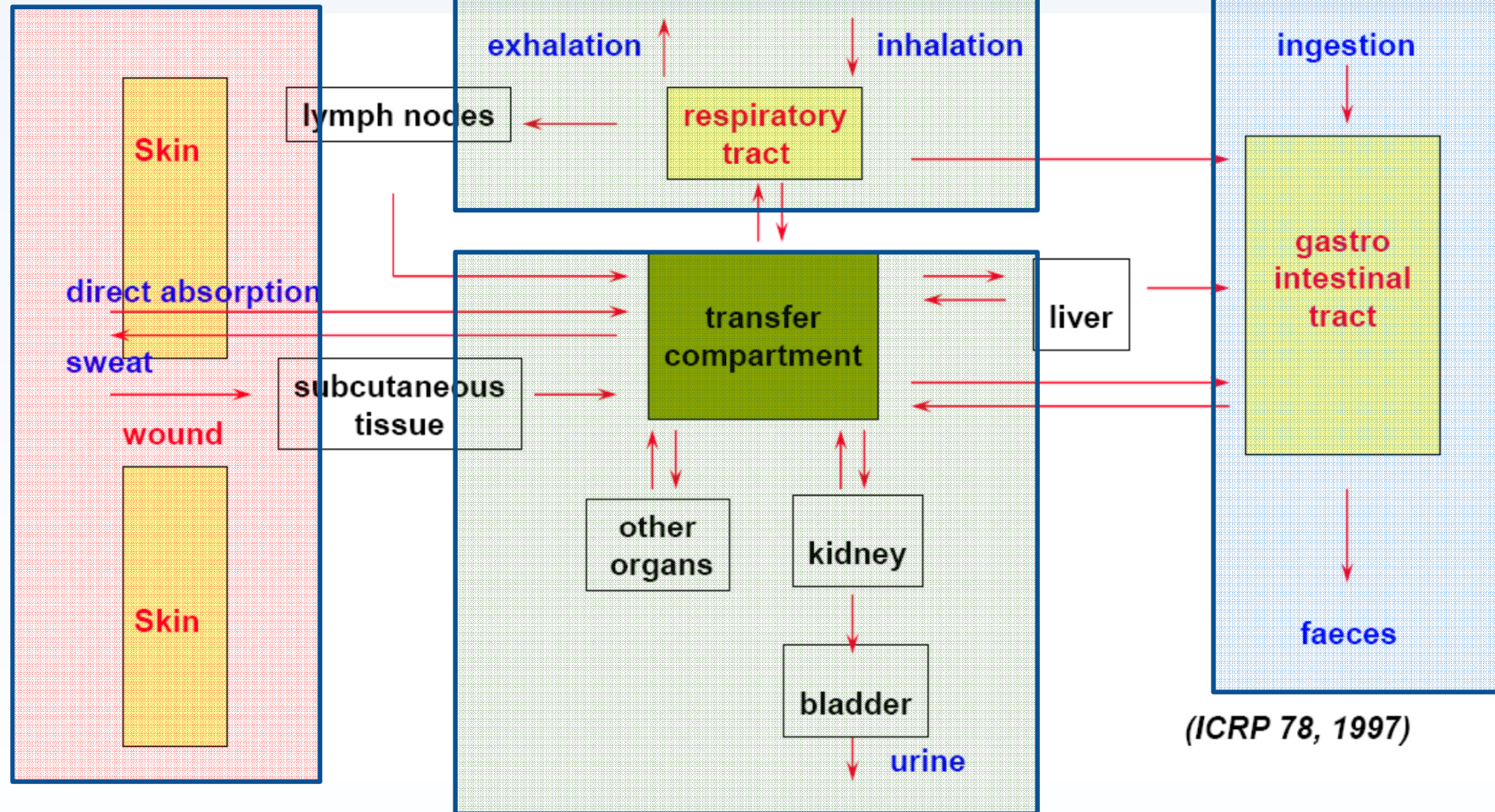
Generic biokinetic model



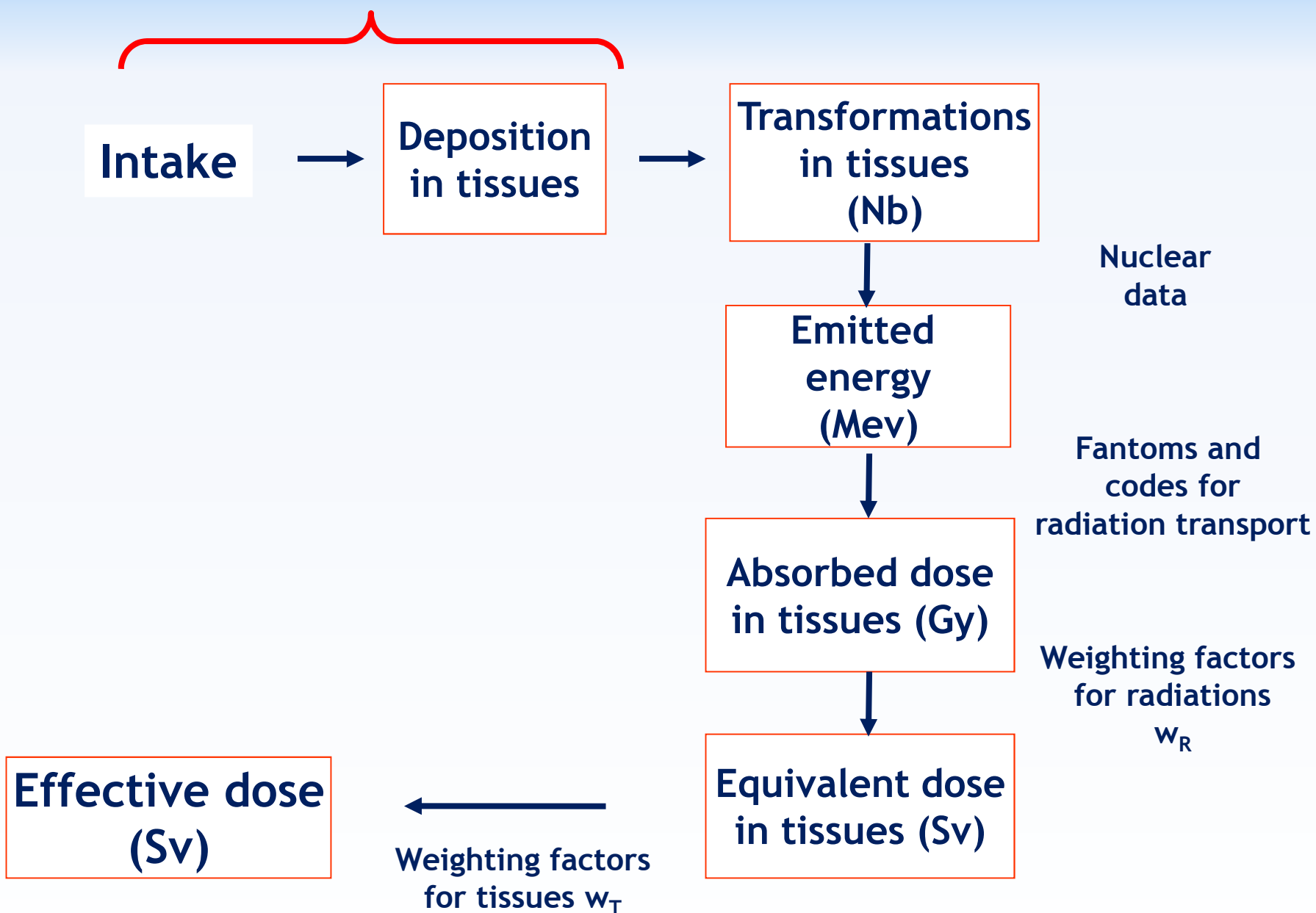
HRTM

HATM

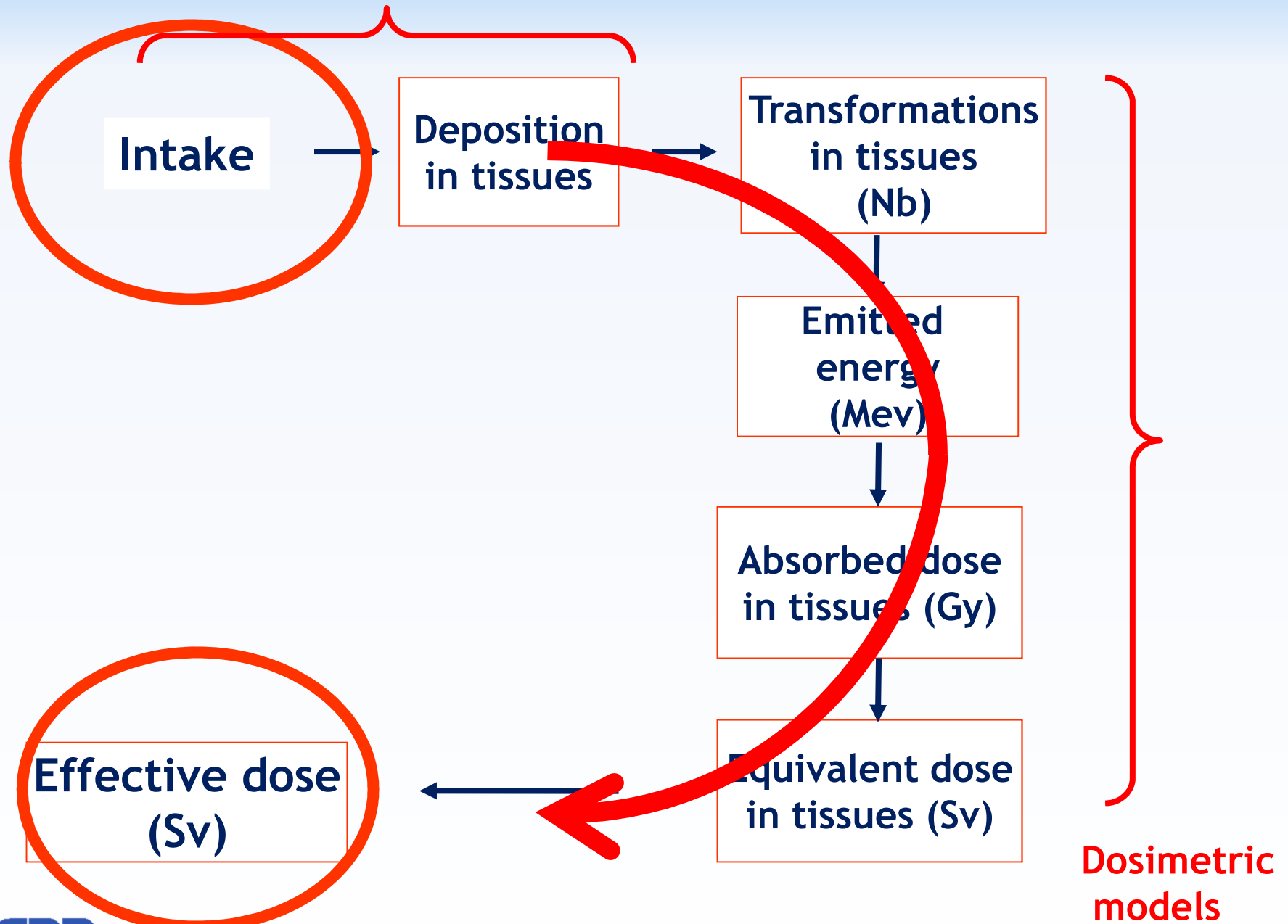
WOUND



Biokinetic models

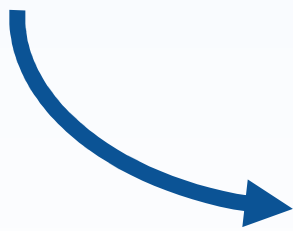


Biokinetic models



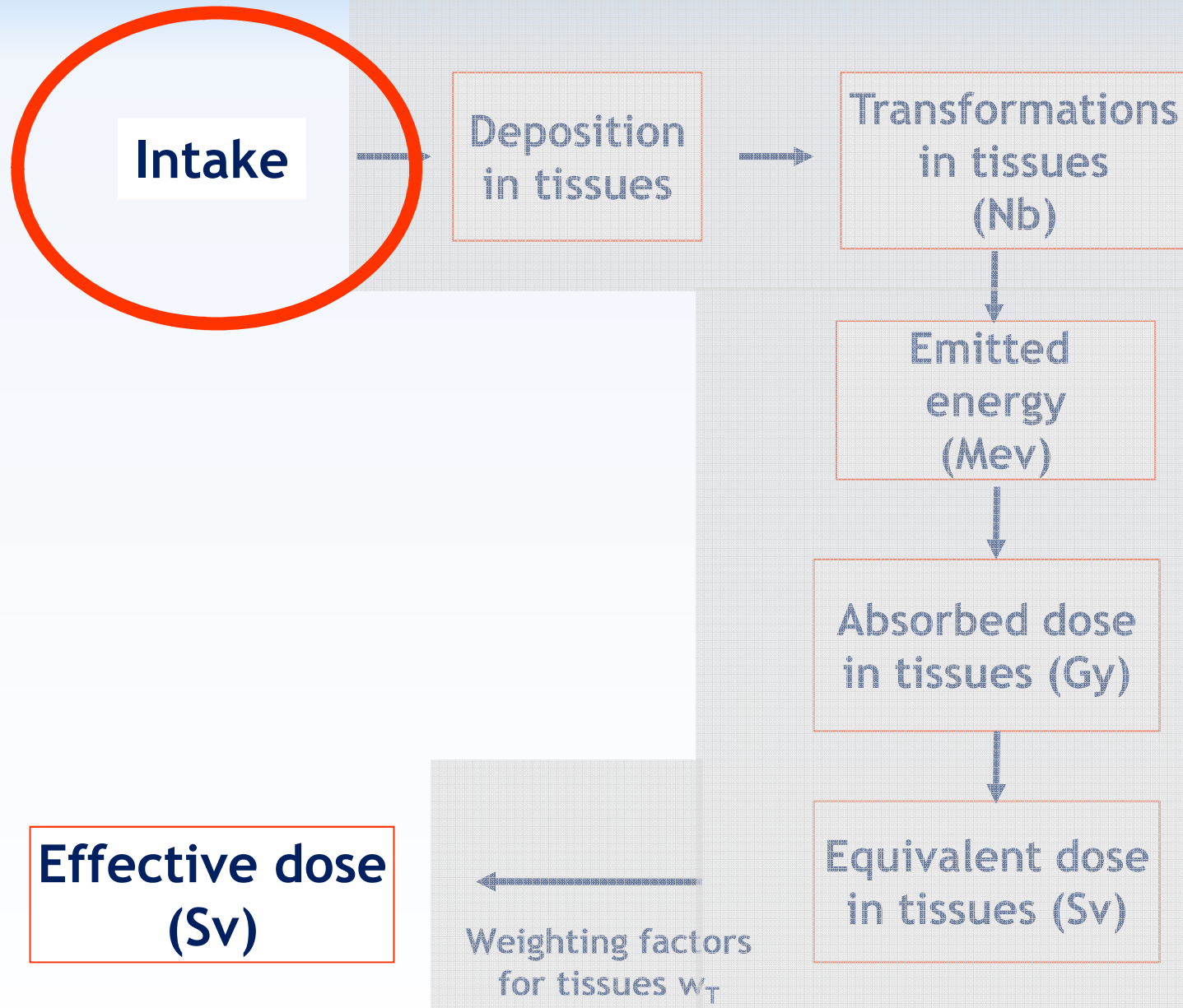
Complex procedure, limited to experts

ICRP has defined concepts and tools, to allow non-specialist to perform dose assessment



1. Dose per unit intake

2. Retention functions



Intake



Deposition
in tissues



Transformations
in tissues
(Nb)



DPUI



Type

F

M

S

DPUI
 $\mu\text{Sv/Bq}$

0.6

2.1

6.8

For ^{234}U

$$\text{Dose} = \text{Intake} \times \text{DPUI}$$

Effective dose
(Sv)



Weighting factors
for tissues w_T

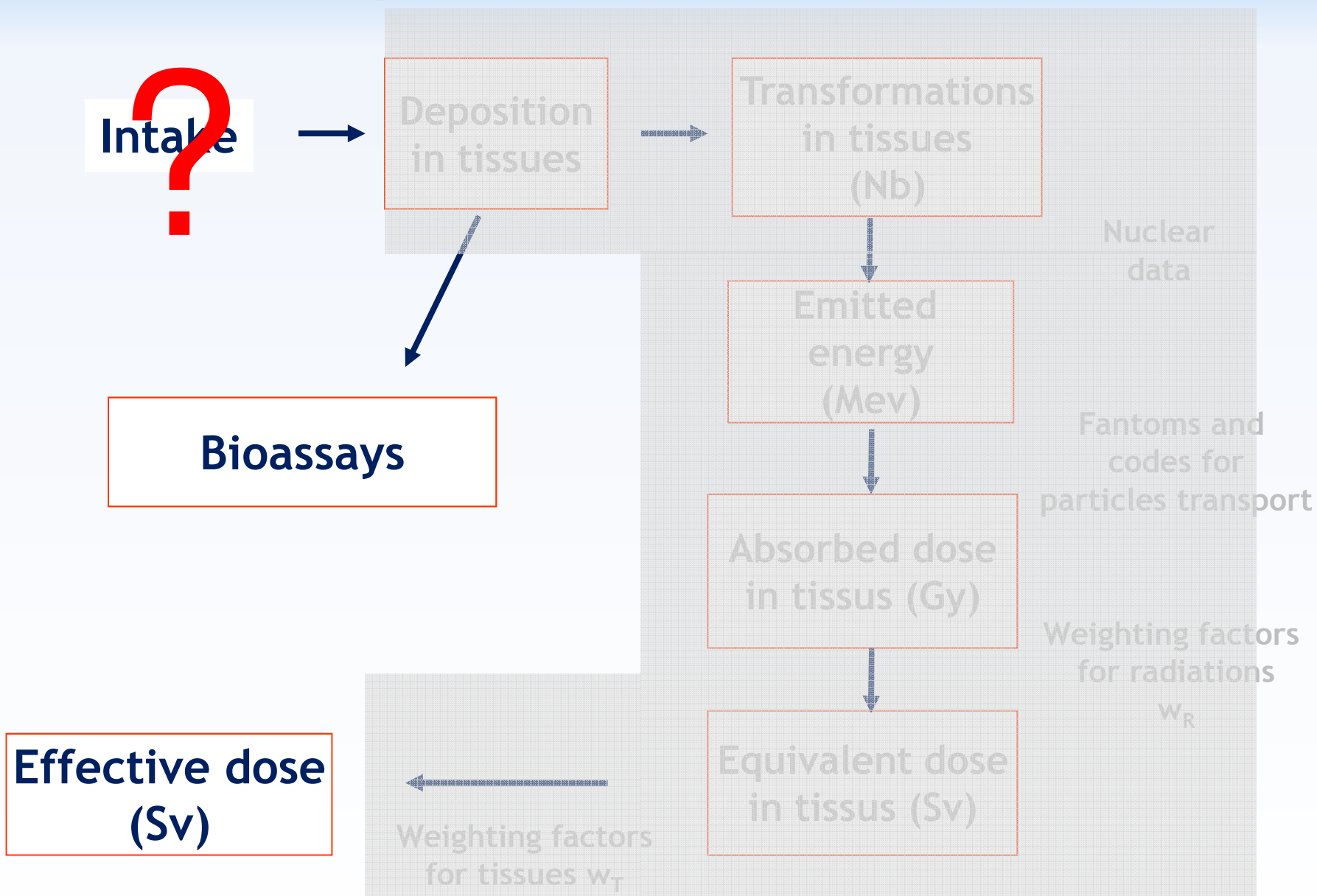
Equivalent dose
in tissues (Sv)

Table B.1.—(continued)

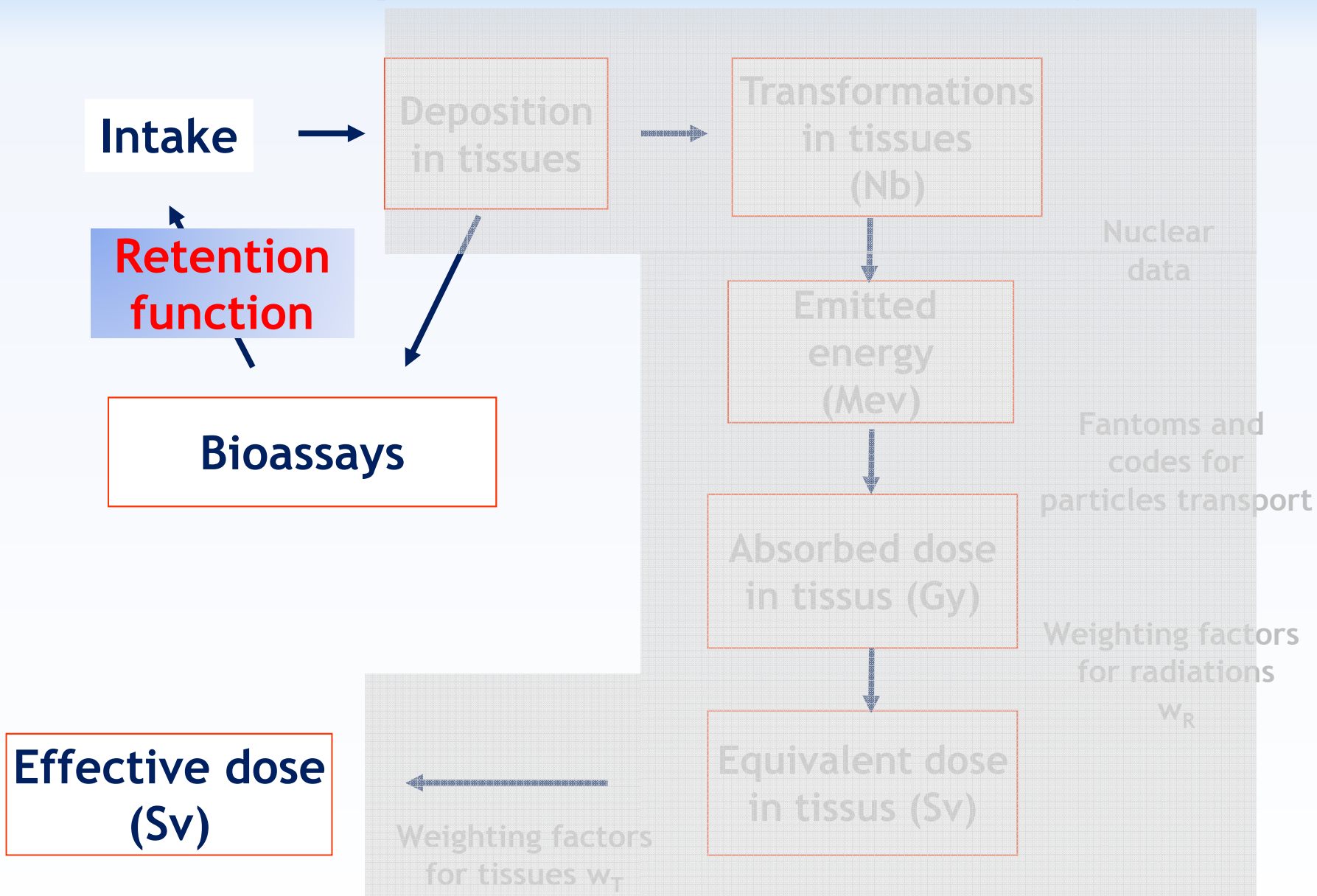
Nuclide	$t_{1/2}$	Effective dose coefficients (Sv Bq ⁻¹)					
		Inhalation, $e_{inh}(50)$				Ingestion	
		Type	f_1	1 μ mAMAD	5 μ mAMAD	f_1	$e_{ing}(50)$
Ca-47	4.53d	M	0.300	1.8E-09	2.1E-09	0.300	1.6E-09
Scandium							
Sc-43	3.89h	S	1.0E-04	1.2E-10	1.8E-10	1.0E-04	1.9E-10
Sc-44	3.93h	S	1.0E-04	1.9E-10	3.0E-10	1.0E-04	3.5E-10
Sc-44m	2.44d	S	1.0E-04	1.5E-09	2.0E-09	1.0E-04	2.4E-09
Sc-46	83.8d	S	1.0E-04	6.4E-09	4.8E-09	1.0E-04	1.5E-09
Sc-47	3.35d	S	1.0E-04	7.0E-10	7.3E-10	1.0E-04	5.4E-10
Sc-48	1.82d	S	1.0E-04	1.1E-09	1.6E-09	1.0E-04	1.7E-09
Sc-49	0.956h	S	1.0E-04	4.1E-11	6.1E-11	1.0E-04	8.2E-11
Titanium							
Ti-44	47.3y	F	0.010	6.1E-08	7.2E-08	0.010	5.8E-09
		M	0.010	4.0E-08	2.7E-08		
		S	0.010	1.2E-07	6.2E-08		
Ti-45	3.08h	F	0.010	4.6E-11	8.3E-11	0.010	1.5E-10
		M	0.010	9.1E-11	1.4E-10		
		S	0.010	9.6E-11	1.5E-10		

From ICRP 68, 1994

General procedures for assessing doses



General procedures for assessing doses



In this case, intake = Bq measured $\times 10^5$

Retention functions

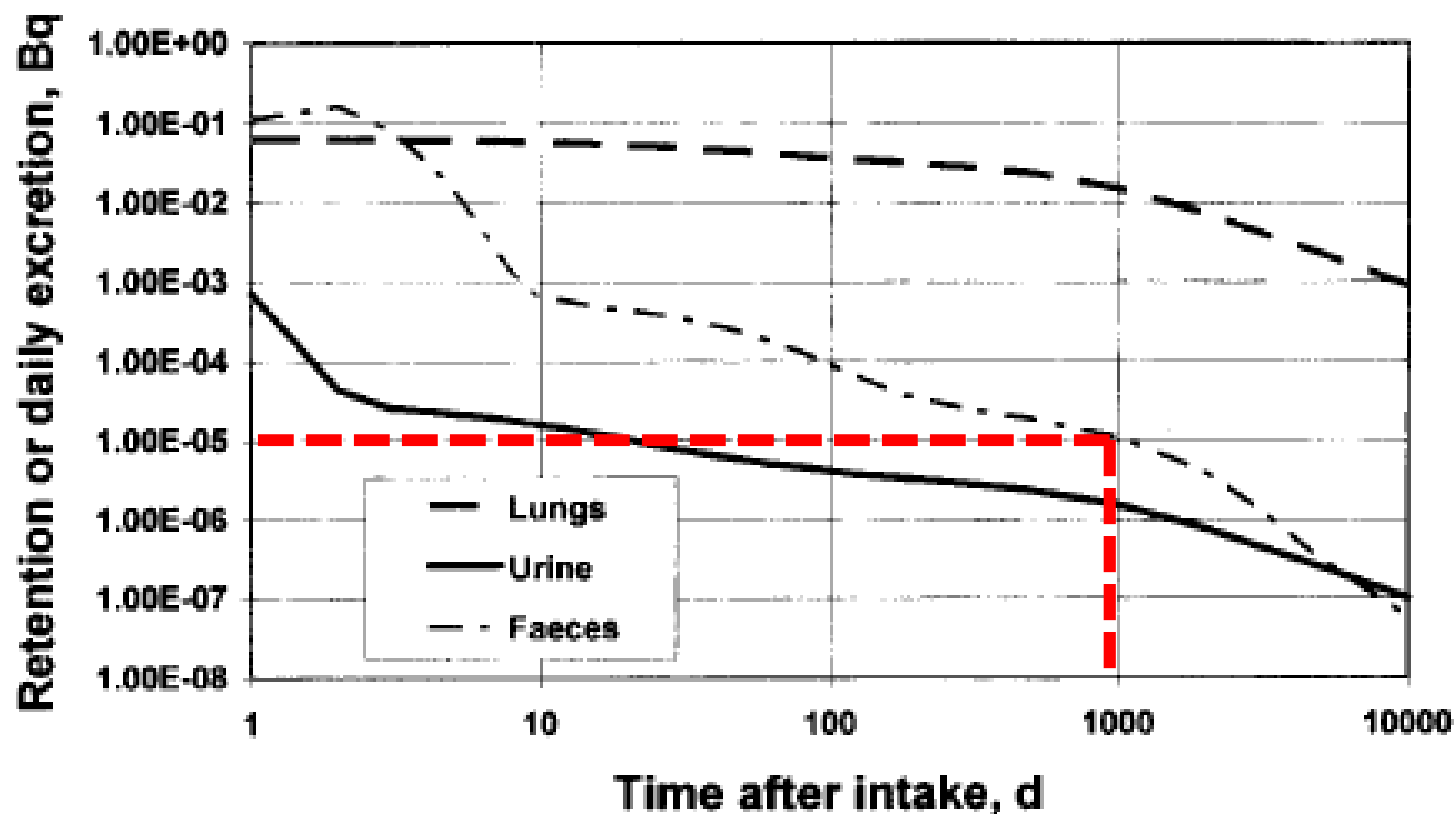
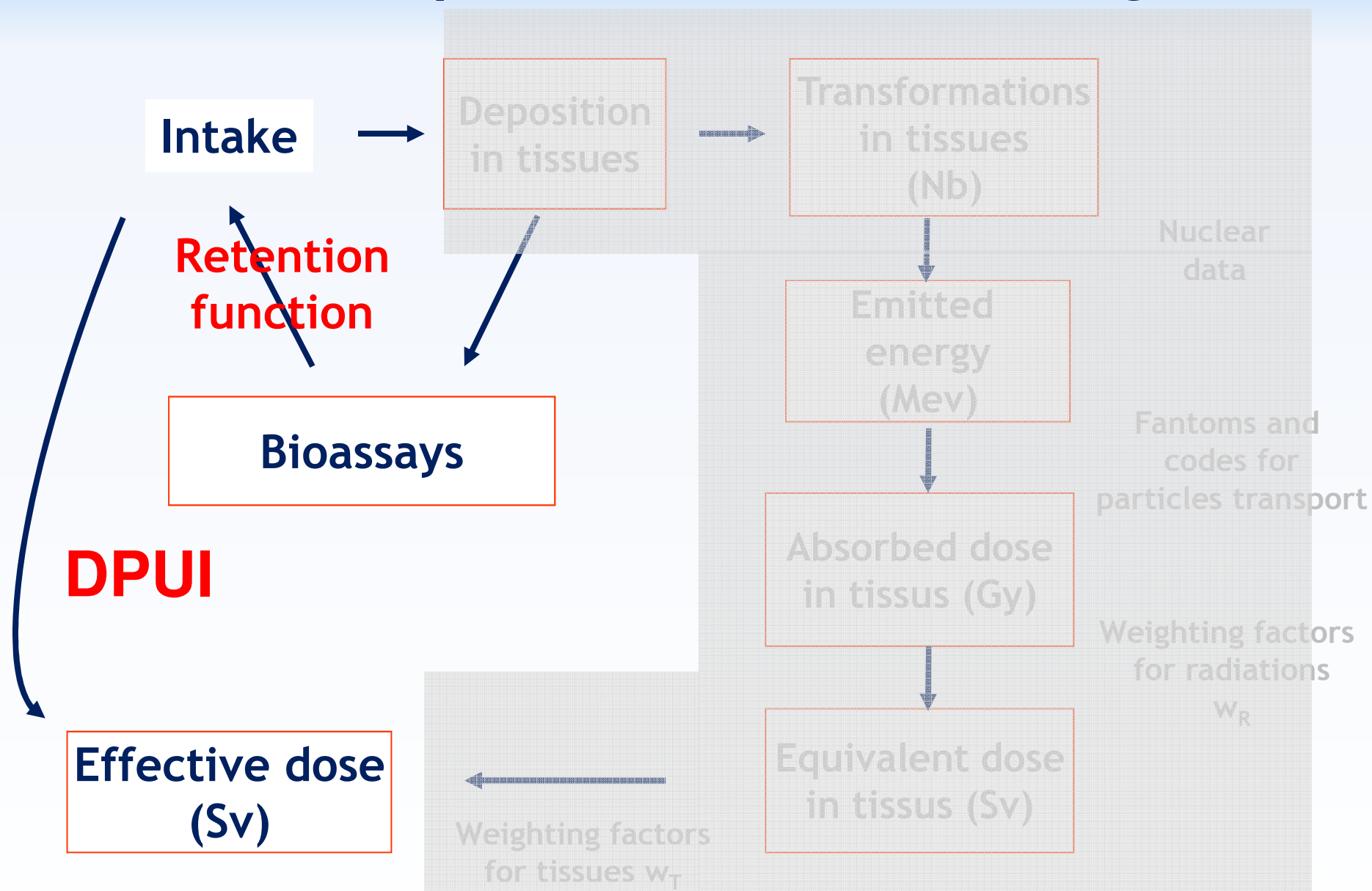


Fig. A.10.3. ^{234}U Inhalation Type S: predicted values (Bq per Bq intake) following acute intake.

General procedures for assessing doses



Main ICRP publications on these topics

For workers

Publication 30 (ICRP, 1979, 1980, 1981, 1988)

dose coefficients and ALI for inhalation and ingestion.
based on Reference man (Publication 23, 1975) and 1977
recommendations (Publication 26, 1977).

Publication 68 (ICRP, 1994)

updated dose coefficients following 1991 Recommendations
(Publication 60, 1991), HRTM (Publication 66, 1994), new skeletal
data (Publication 70, 1995) and revised systemic biokinetic models.
No ALI anymore.

Publications 54 and 78 (ICRP, 1988, 1997)

guidance on the design of monitoring programs and the interpretation
of results, to estimate doses to workers following radionuclide inhalation
or ingestion. Provide predicted values of measured quantities after
intake.

Main ICRP publications on these topics

For the members of the public

Publications 56, 67, 69, 71 and 72 (ICRP, 1989, 1993, 1995)

age-specific dose coefficients for inhalation and ingestion for 91 elements, using up-to-date models and latest ICRP recommendations.

Publications 88 and 95 (2001,2004)

Dose to embryo/fetus and infants

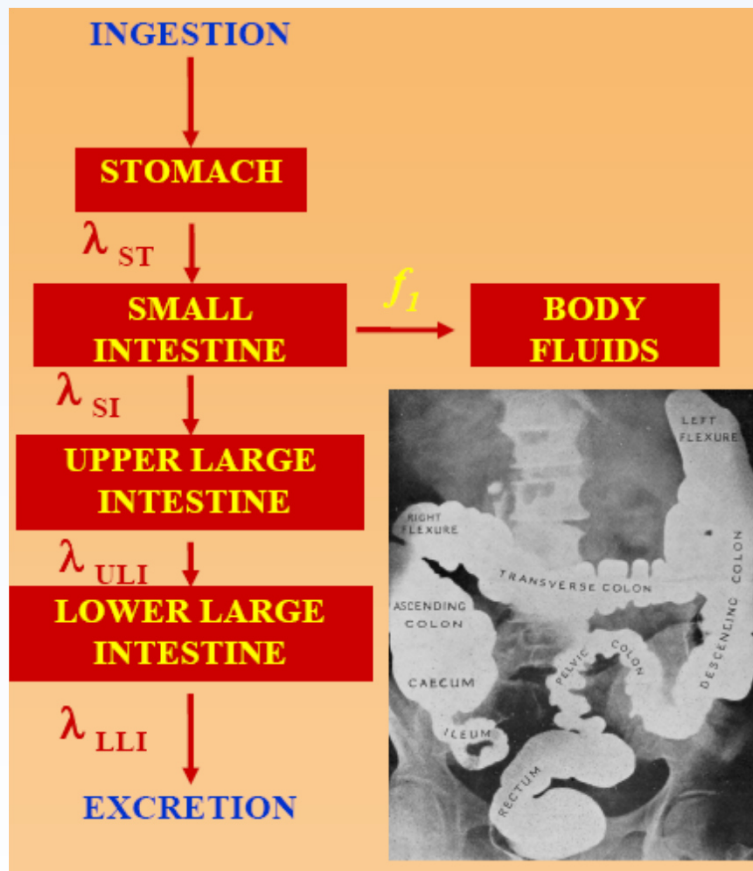
Progress and changes made during this period

In physiology and biokinetic models

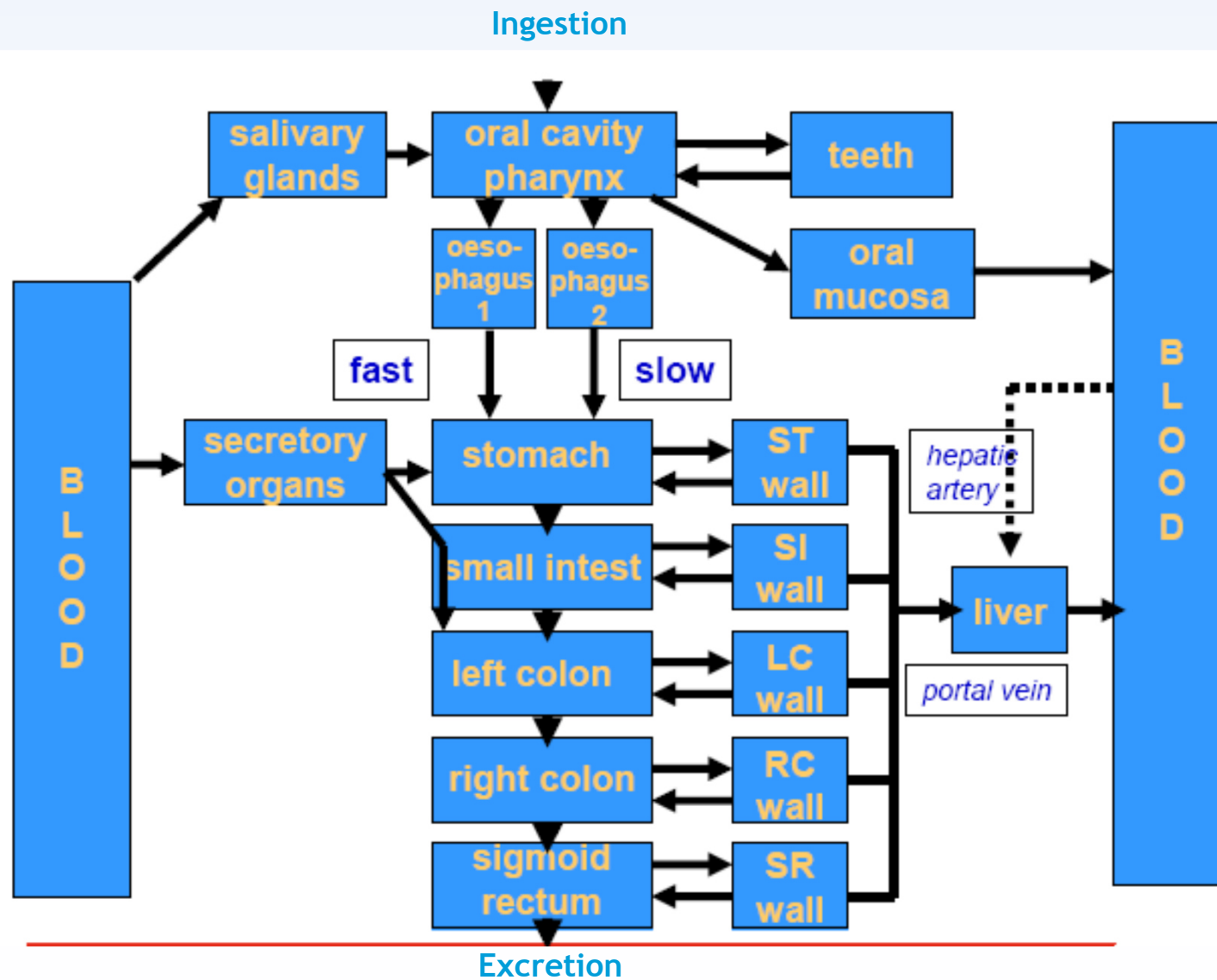
- New data on Reference man (ICRP 89, 2002)
- Human Alimentary Tract Model (ICRP 100, 2006)

The Human alimentary tract model

The former model



The Human alimentary tract model

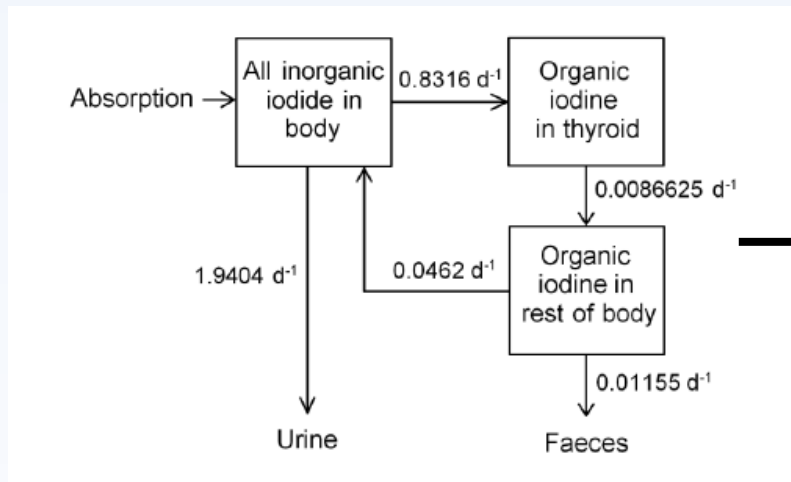


Progress and changes made during this period

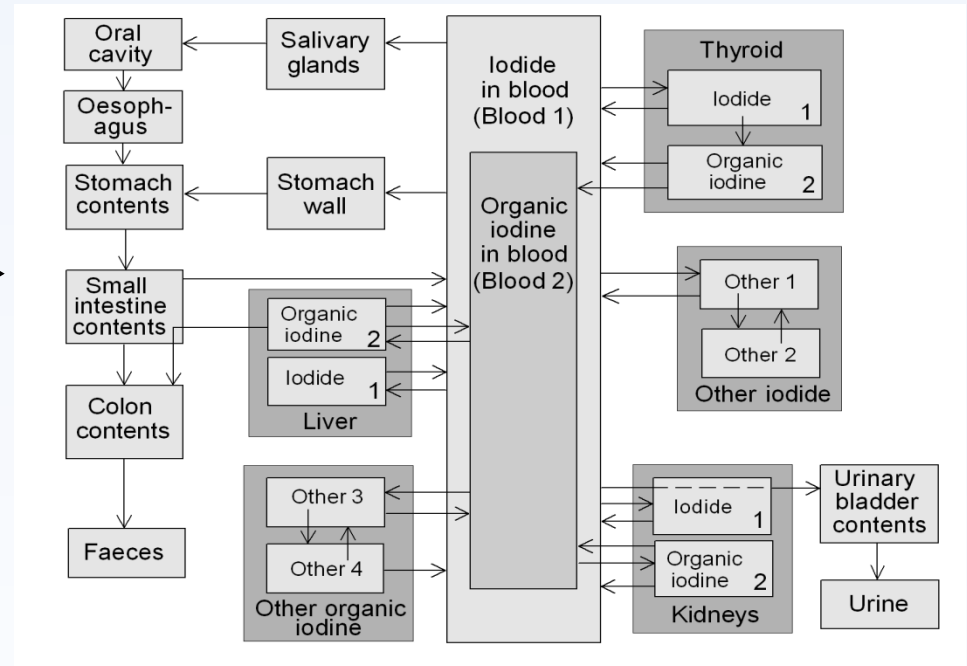
In physiology and biokinetic models

- New data on Reference man (ICRP 89, 2002)
- Human Alimentary Tract Model (ICRP 100, 2006)
- New element specific systemic models, physiologically realistic

Systemic model for Iodine



The former model (ICRP 1994, 1997)



The new model

Three subsystems:

- circulating inorganic iodide;
- thyroidal organic iodine
- extrathyroidal organic iodine.

Systemic model for Strontium

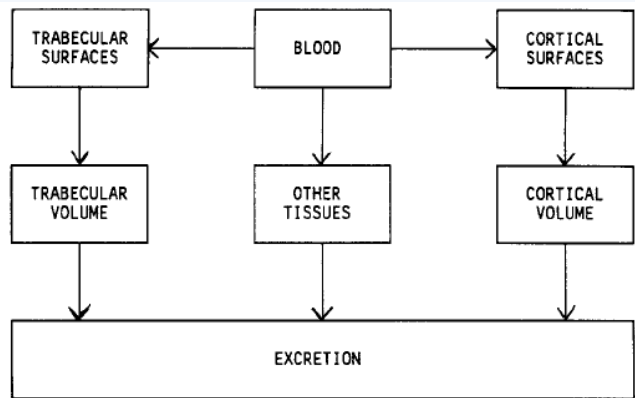


Fig. 2. Diagram of the biokinetic model for strontium.

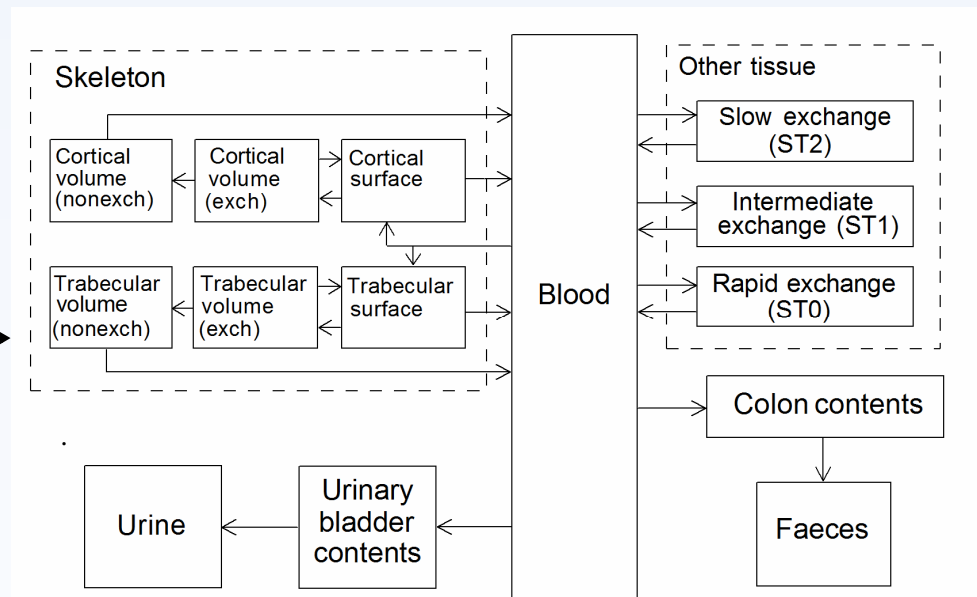


Figure 10-1. Structure of the biokinetic model for systemic strontium.

Abbreviations: exch = exchangeable, nonexch = non-exchangeable

The former model (ICRP 1989)

The new model

Systemic model for Radon

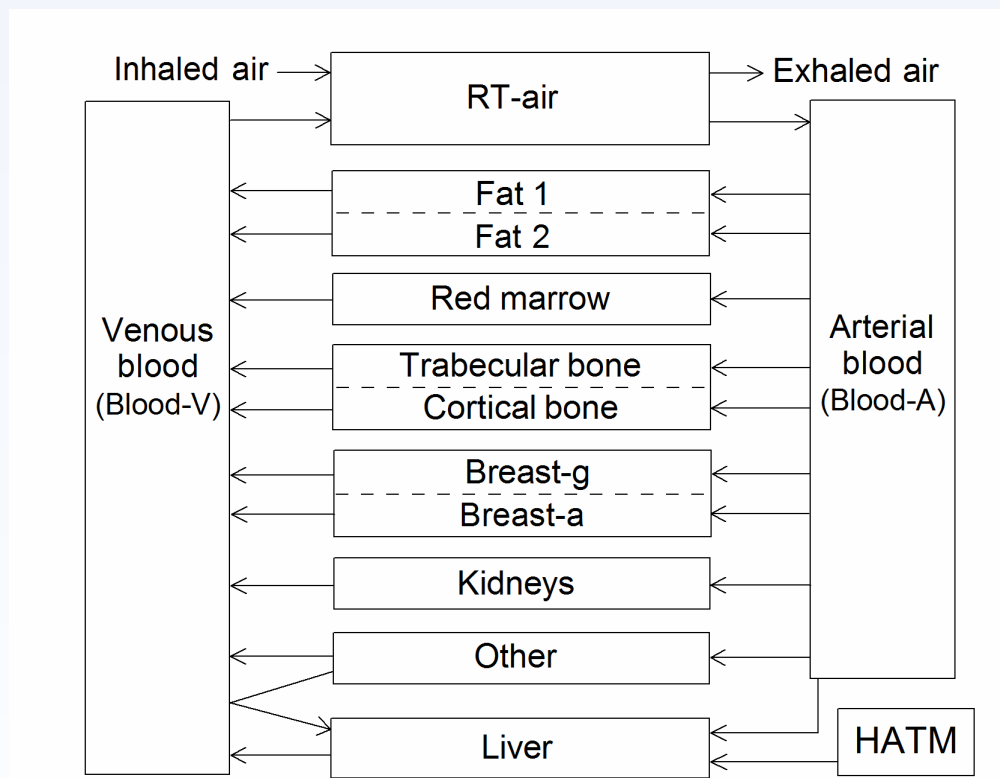


Figure 12.6. Structure of the biokinetic model for systemic radon.

Abbreviations: RT-air = respiratory tract air; Blood-A = arterial blood; Blood-V = venous blood;

Breast-g = glandular breast tissue; Breast-a = adipose tissue in breast;

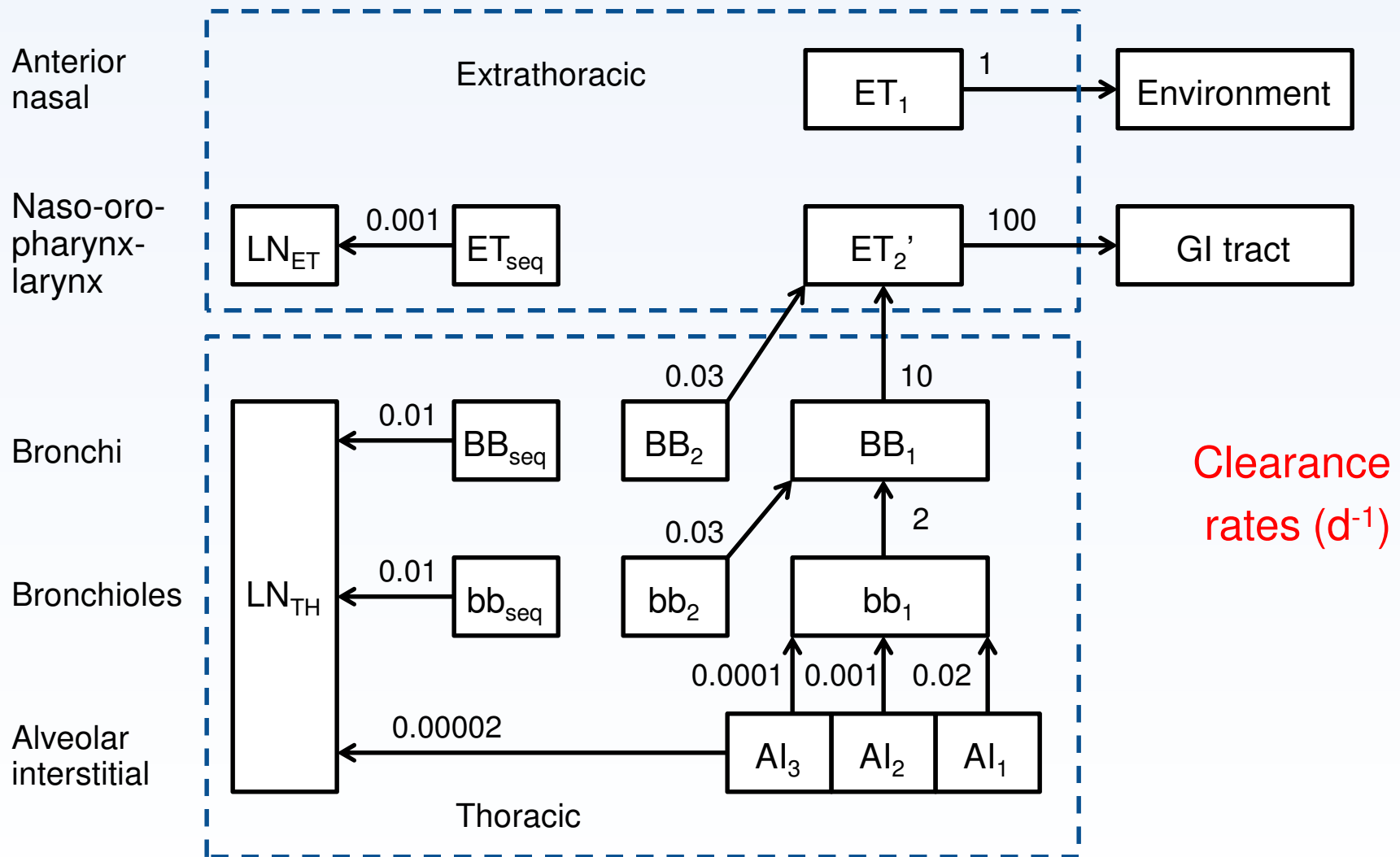
HATM = Human Alimentary Tract Model (ICRP, 2006).

Progress and changes made during this period

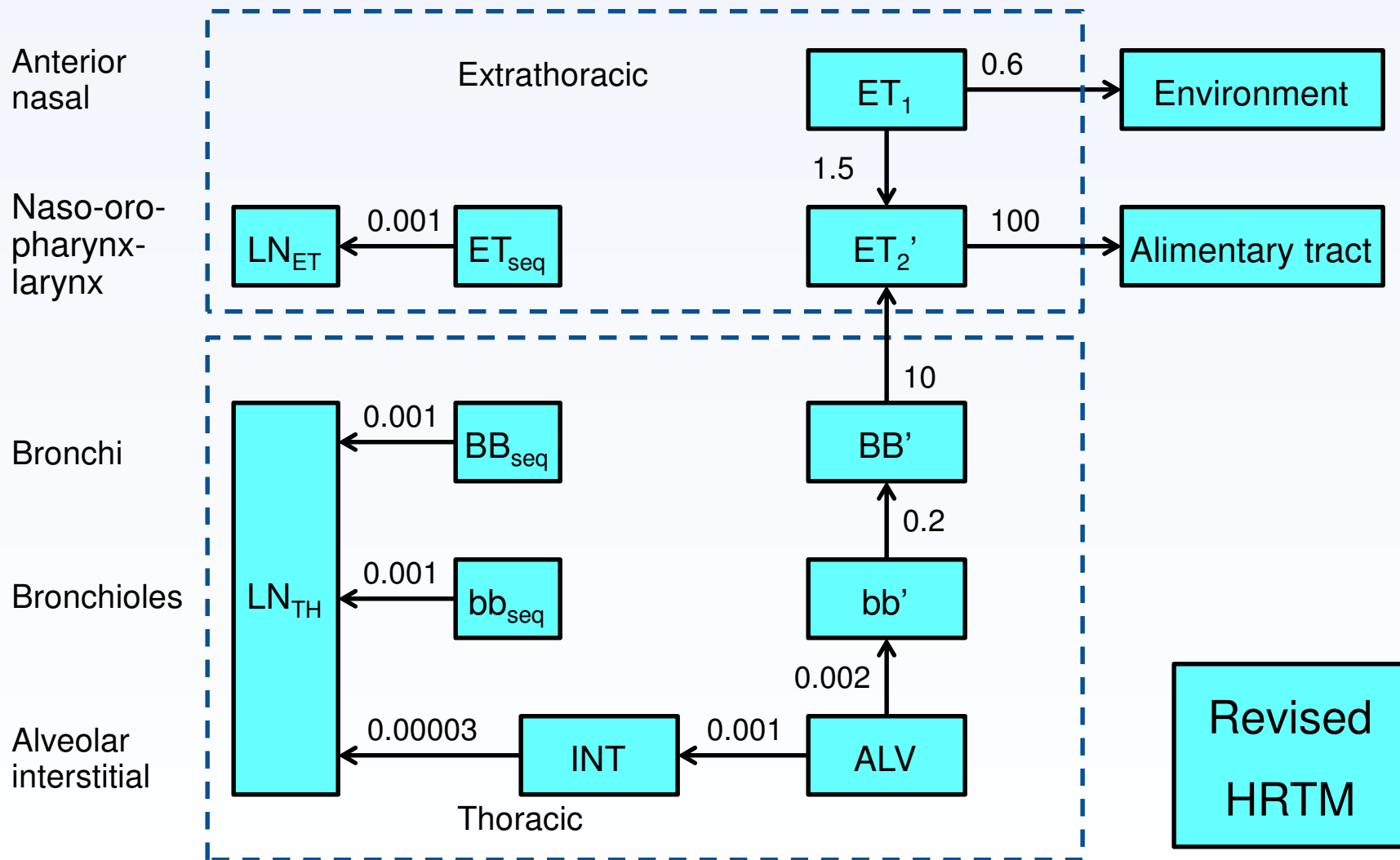
In physiology and biokinetic models

- New data on Reference man (ICRP 89, 2002)
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- More realistic treatment of the biokinetics of radionuclide daughters
- New data supporting update of the Human Respiratory Tract Model

Particle transport model (ICRP 66 HRTM)



Particle transport model (ICRP 103)



Uranium absorption

Compound	Absorption parameter values			Type
	f_r	s_r (d ⁻¹)	s_s (d ⁻¹)	
Default Type F	1.0	100		

Element-specific values for f_r , s_r and s_s

Uranium absorption

Compound	Absorption parameter values			Type
	f_r	s_r (d ⁻¹)	s_s (d ⁻¹)	
Default Type F	1.0	100		
Uranyl nitrate, UO ₂ (NO ₃) ₂	0.9	3	0.005	(F)
U-Tri-butyl-phosphate	0.97	12	0.002	(F)
Uranium peroxide hydrate	0.9	0.9	0.024	(F)

Element-specific values for f_r , s_r and s_s

Uranium absorption

Compound	Absorption parameter values			Type
	f_r	s_r (d ⁻¹)	s_s (d ⁻¹)	
Default Type F	1.0	100		
Uranyl nitrate, UO ₂ (NO ₃) ₂	0.9	3	0.005	(F)
U-Tri-butyl-phosphate	0.97	12	0.002	(F)
Uranium peroxide hydrate	0.9	0.9	0.024	(F)
Default Type M	0.1	100	0.005	

Element-specific values for f_r , s_r and s_s

Uranium absorption

Compound	Absorption parameter values			Type
	f_r	s_r (d ⁻¹)	s_s (d ⁻¹)	
Default Type F	1.0	100		
Uranyl nitrate, UO ₂ (NO ₃) ₂	0.9	3	0.005	(F)
U-Tri-butyl-phosphate	0.97	12	0.002	(F)
Uranium peroxide hydrate	0.9	0.9	0.024	(F)
Default Type M	0.1	100	0.005	
Ammonium diuranate, ADU	0.8	0.7	0.02	(M)
Uranium tetrafluoride	0.6	0.15	0.005	(M)
Uranium trioxide	0.8	1	0.01	(M)
U ₃ O ₈	0.04	1	0.004	(M)

Element-specific values for f_r , s_r and s_s

Uranium absorption

Compound	Absorption parameter values			Type
	f_r	s_r (d ⁻¹)	s_s (d ⁻¹)	
Default Type F	1.0	100		
Uranyl nitrate, UO ₂ (NO ₃) ₂	0.9	3	0.005	(F)
U-Tri-butyl-phosphate	0.97	12	0.002	(F)
Uranium peroxide hydrate	0.9	0.9	0.024	(F)
Default Type M	0.1	100	0.005	
Ammonium diuranate, ADU	0.8	0.7	0.02	(M)
Uranium tetrafluoride	0.6	0.15	0.005	(M)
Uranium trioxide	0.8	1	0.01	(M)
U ₃ O ₈	0.04	1	0.004	(M)
Default Type S	0.001	100	0.0001	
Uranium dioxide	0.015	1	0.0005	(S)

Element-specific values for f_r , s_r and s_s

Default parameter values

Type F, M, S

		f_r	s_r (d ⁻¹)	s_s (d ⁻¹)
Type F	ICRP 66 <i>OIR</i>	1 1	100 30	
Type M	ICRP 66 <i>OIR</i>	0.1 0.2	100 3	0.005 0.005
Type S	ICRP 66 <i>OIR</i>	0.001 0.01	100 3	0.0001 0.0001

Progress and changes made during this period

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In dosimetry and monitoring

- Development of adult reference computational phantom, based on the new ref man (ICRP 110, 2009)
- New skeletal dosimetry (ICRP 116, 2010)
- Revised nuclear decay data (ICRP 107, 2008)

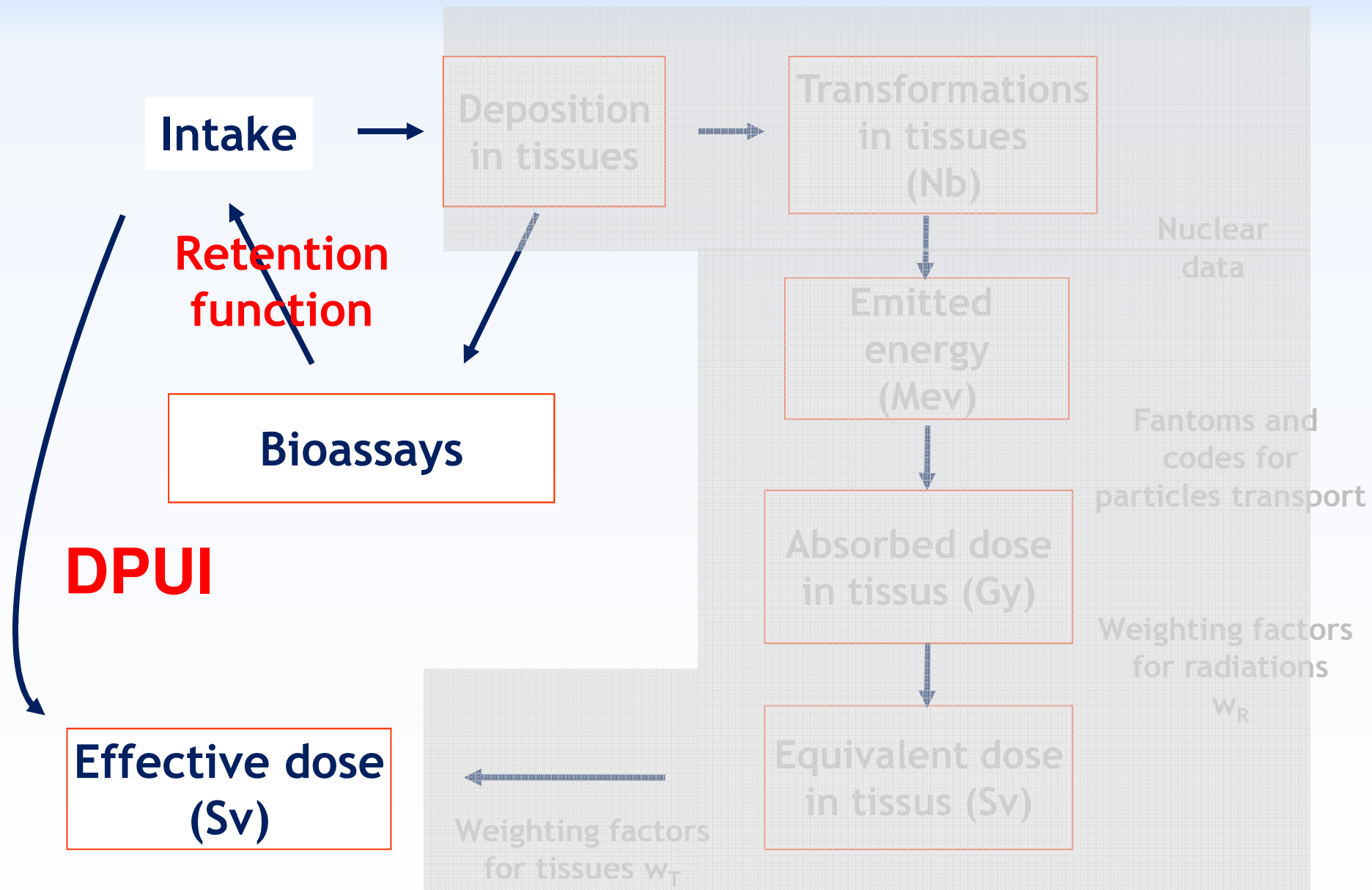
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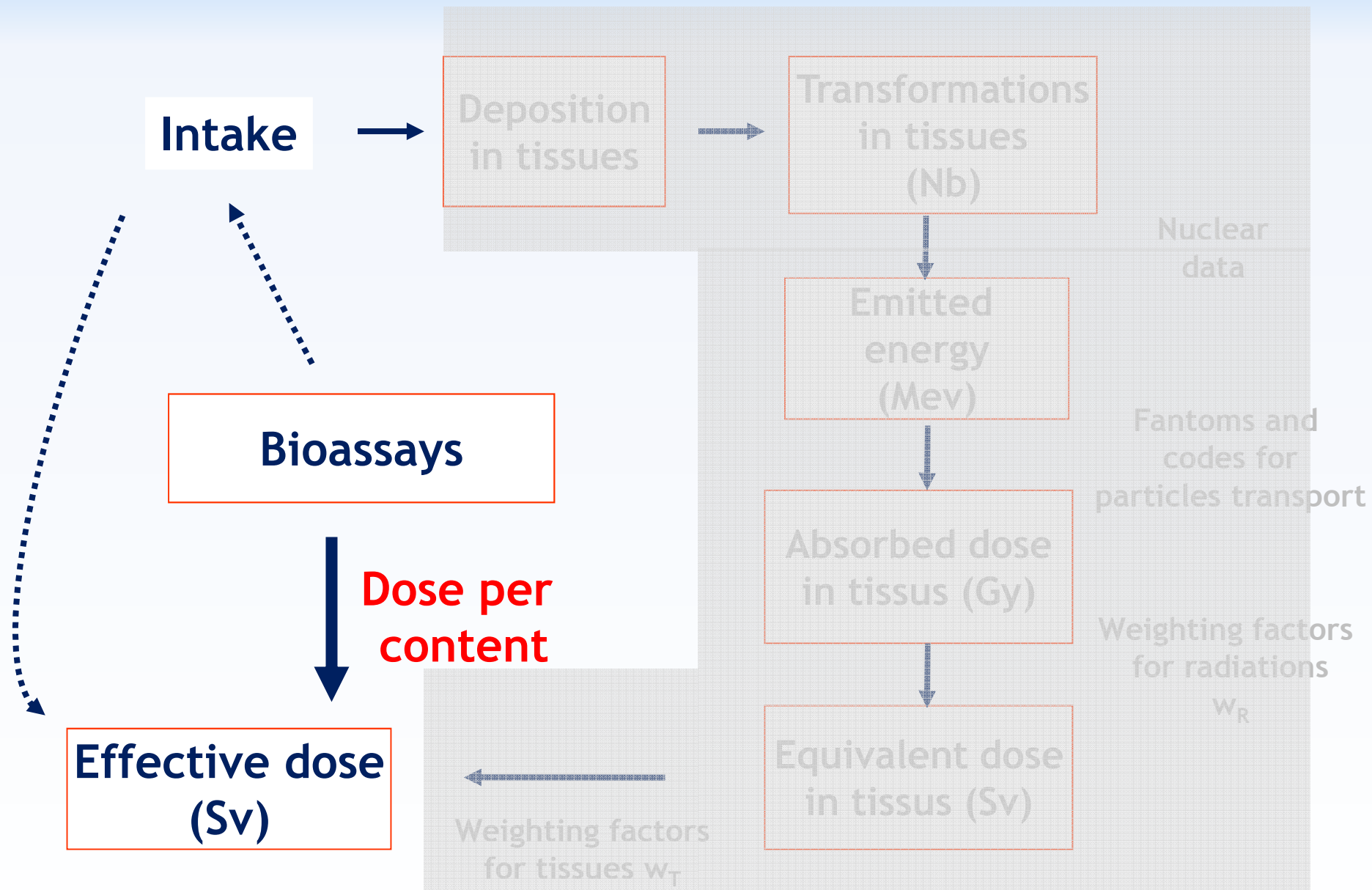
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In dosimetry and monitoring

- Development of adult reference computational phantom, based on the new ref man (ICRP 110, 2009)
- New skeletal dosimetry (ICRP 116, 2010)
- Revised nuclear decay data (ICRP 107, 2008)
- Concept of dose per content



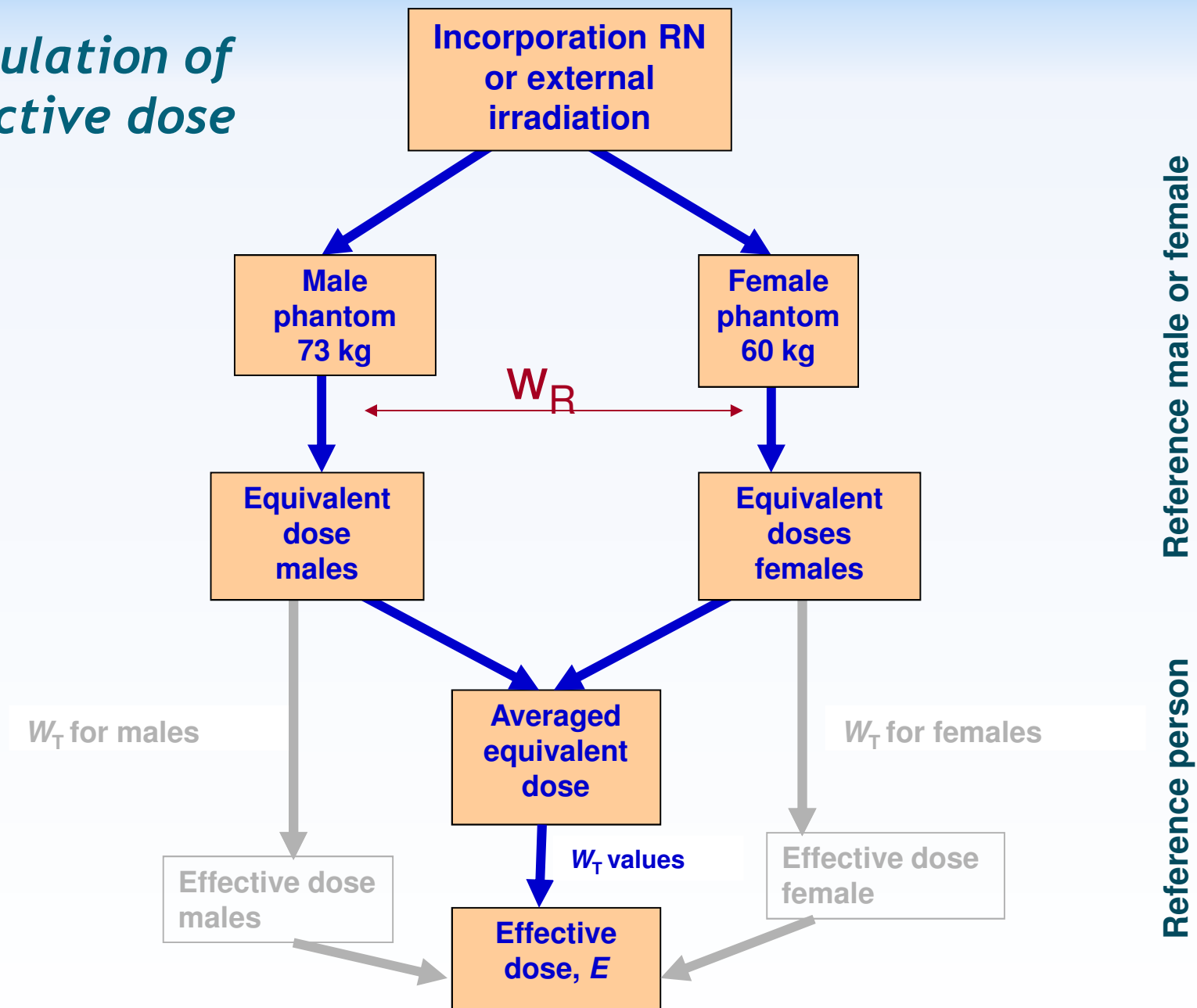


Progress and changes made during this period (con't)

In ICRP recommendations

- Adoption of the use of realistic phantoms (ICRP 103, 2007)
- Changes in weighting factors (ICRP 103, 2007)
- Changes in calculation of equivalent dose (ICRP 103, 2007)

Calculation of effective dose



Progress and changes made during this period (con't)

In ICRP recommendations

- Adoption of the use of realistic phantoms (ICRP 103, 2007)
- Changes in weighting factors (ICRP 103, 2007)
- Changes in calculation of equivalent dose (ICRP 103, 2007)
- Need to provide dose coefficients for Radon (ICRP 2009)

Progress and changes made during this period (con't)

These new data and recommendations supported a revision of the past reports and provision of new dose coefficients with guidance on monitoring programs and data interpretation

**Done for external dosimetry (ICRP 116, 2010)
Need to be done for internal dosimetry**

Revision of the reports on internal exposure

Division of the work in two parts :

- Revision of models and dose coefficients for workers
(OIR series)
- Revision of models and dose coefficients for members of the public *(Age dependant series, Embryo and fetus, maternal transfer,..)*

The OIR series 5 volumes

OIR Part 1 (ICRP Publication 130)

- Control of occupational exposures to radionuclides
- Biokinetic and dosimetric models
- Methods of individual and workplace monitoring
- Monitoring programmes
- General aspects of retrospective dose assessment

To be published today!!

The OIR series 5 volumes

OIR Part 2 to 5

For each element section:

- **Chemical forms in the workplaces**
- **Principal radioisotopes, physical half-lives and decay modes**
- **Review of data on inhalation, ingestion and systemic biokinetics**
- **Structure of biokinetic models and parameter values**
- **Monitoring techniques and typical detection limits**
- **Dose coefficients, reference bioassays functions and dose per content functions in printed document and/or electronic annexes**

The OIR series

5 volumes

OIR Part 2

Hydrogen (H), Carbon (C), Phosphorus (P), Cobalt (Co), Zinc (Zn), Strontium (Sr), Yttrium (Y), Molybdenum (Mo) and Technetium (Tc).

2016

Calcium (Ca), Iron (Fe), Zirconium (Zr), Niobium (Nb),

OIR Part 3

Ruthenium (Ru), Antimony (Sb), Tellurium (Te), Iridium (Ir), Lead (Pb), Bismuth (Bi), Polonium (Po), Thorium (Th) and Uranium (U).

2016

Cesium (Cs), Barium (Ba), Radon (Rn), Radium (Ra),

OIR Part 4

Lanthanides series, actinium (Ac), protactinium (Pa),

2017

transuranic elements

OIR Part 5

Fluorine (F), Sodium (Na), Magnesium (Mg), Potassium (K), Manganese (Mn), Nickel (Ni), Selenium (Se), Molybdenum (Mo), Technetium (Tc) and Silver (Ag) and



most of the others

INTERNATIONAL COMMISSION ON RADIOLOGICAL PROTECTION

Uncertainty and variability in dose calculation

**Uncertainty : lack of knowledge of a central value
for a population**

**Variability : difference between members
result from physiological or
environmental factors**

Variability may induce uncertainty !!

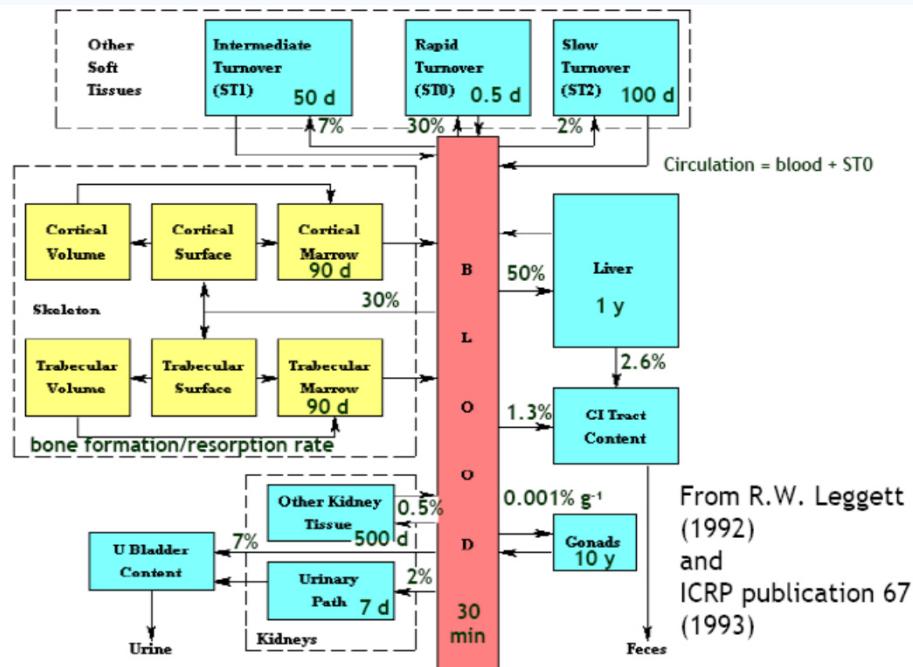
Uncertainty in dose calculation

Depends on uncertainties :

- **in measurement of activity**
- **in exposure scenario (route, time, RN, form,..)**
- **in biokinetic and dosimetric models**

Uncertainties in biokinetic models

on parameter values AND on model structure



Uncertainties in biokinetic models

Depends on available information to derive biokinetic models

- **Human data**
(available for Essent. elts + Cs, Pb, Ra, U, Am, Pu,..)

Uncertainties in biokinetic models

- Human data
- **Human data on chemically similar elements (Ln)**

Uncertainties in biokinetic models

- Human data
- **Human data on chemically similar elements (Ln)**

Ca, Sr, Ba, Ra chemically and physiologically similar.. but
Na and K chemically similar but physiologically different
Cs and K chemically similar but different kinetics

Uncertainties in biokinetic models

- Human data
- Human data on chemically similar elements
- **Animal data**

Need extrapolation



Uncertainties in biokinetic models

- Human data
- Human data on chemically similar elements
- Animal data
- **Animal data AND chemically similar elements**

There are many sources of uncertainty but :

Reference models and parameter values are fixed by convention and are not subject to uncertainty

They are intended to be used for optimisation and demonstration of compliance with dose limits

In case where models are used for other scientific purposes, uncertainties and variability may be taken into account.

Conclusions

Determination of internal doses is a complex procedure

Many tools available to directly assess dose from activity measurements

Based on the use of biokinetic and dosimetric models

No uncertainty for calculation of effective dose

www.ICRP.org